

Selected Tables and Figures from the
Final Report of the Be-9 Evaluation

„Evaluation of all important neutron cross
sections for ^9Be and evaluation of the
secondary neutrons from the interaction of
neutrons with Be in the neutron energy range
from 10^{-5} to 20 MeV“

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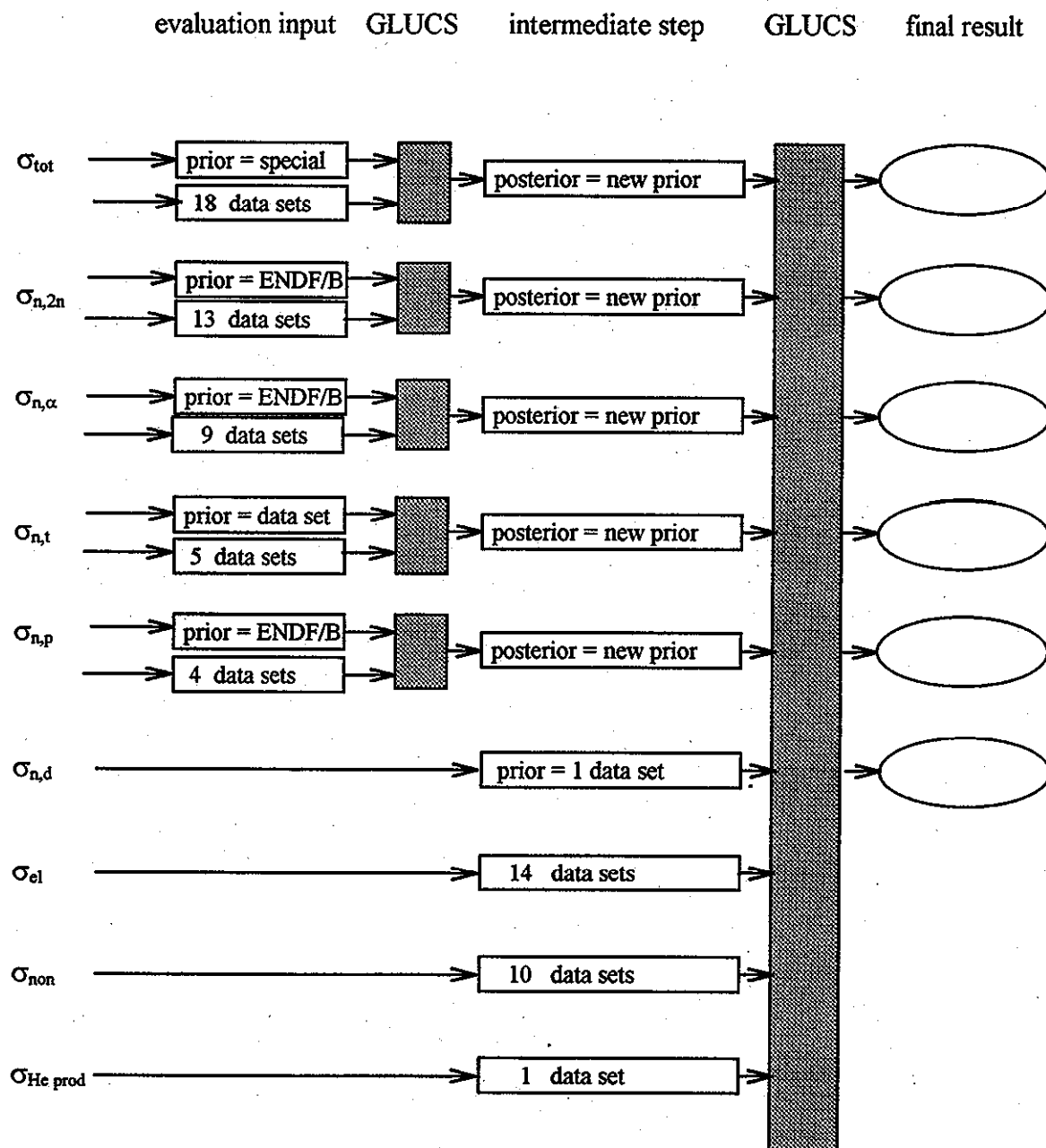
(in preparation)

JEF/DOC-635

≡ EFF-Doc-527

⁹Be evaluation-flow chart

(integral cross sections)



MT = 1, printed from totstep2.dat

	Neutron energy (in MeV)	cross section (in barn)	std.dev. of sigma(E) (in barn)
1	2.5000E-01	4.4672E+00	4.190E-02
2	7.5000E-01	3.7471E+00	2.841E-02
3	1.2500E+00	2.7076E+00	1.639E-02
4	1.7500E+00	1.7969E+00	9.701E-03
5	2.2500E+00	1.9534E+00	9.011E-03
6	2.7500E+00	3.3882E+00	1.695E-02
7	3.2500E+00	2.6300E+00	1.406E-02
8	3.7500E+00	2.0982E+00	1.070E-02
9	4.2500E+00	1.9279E+00	9.878E-03
10	4.7500E+00	1.8976E+00	9.901E-03
11	5.2500E+00	1.8627E+00	8.596E-03
12	5.7500E+00	1.8485E+00	7.564E-03
13	6.2500E+00	1.8097E+00	7.539E-03
14	6.7500E+00	1.7744E+00	6.725E-03
15	7.2500E+00	1.7443E+00	7.279E-03
16	7.7500E+00	1.7361E+00	7.450E-03
17	8.2500E+00	1.7207E+00	7.651E-03
18	8.7500E+00	1.7066E+00	7.852E-03
19	9.2500E+00	1.6918E+00	7.515E-03
20	9.7500E+00	1.6802E+00	7.235E-03
21	1.0250E+01	1.6595E+00	7.076E-03
22	1.0750E+01	1.6416E+00	7.693E-03
23	1.1250E+01	1.6181E+00	7.213E-03
24	1.1750E+01	1.5888E+00	7.714E-03
25	1.2250E+01	1.5645E+00	7.031E-03
26	1.2750E+01	1.5534E+00	6.824E-03
27	1.3250E+01	1.5248E+00	6.517E-03
28	1.3750E+01	1.5054E+00	6.850E-03
29	1.4250E+01	1.4905E+00	6.954E-03
30	1.4750E+01	1.4670E+00	7.828E-03
31	1.5500E+01	1.4423E+00	7.956E-03
32	1.6500E+01	1.4255E+00	7.673E-03
33	1.7500E+01	1.3942E+00	6.375E-03
34	1.8500E+01	1.3494E+00	8.064E-03
35	1.9500E+01	1.3211E+00	9.170E-03
36	2.0000E+01	1.3211E+00	9.170E-03

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22650003

MT = 2, printed from be9mt2.dat

	Neutron energy (in MeV)	cross section (in barn)	std.dev. of sigma(E) (in barn)
1	below	= total cross section	
2	7.7500E-01	3.6951E+00	2.835E-02
3	9.2500E-01	3.3830E+00	2.597E-02
4	1.1000E+00	3.0185E+00	2.322E-02
5	1.3000E+00	2.6112E+00	1.598E-02
6	1.5000E+00	2.2384E+00	1.389E-02
7	1.7000E+00	1.8632E+00	1.181E-02
8	1.9000E+00	1.8061E+00	1.027E-02
9	2.1000E+00	1.8524E+00	1.076E-02
10	2.3000E+00	2.0212E+00	1.023E-02
11	2.5000E+00	2.5706E+00	1.294E-02
12	2.7000E+00	3.1154E+00	1.587E-02
13	2.9000E+00	2.7903E+00	2.154E-02
14	3.2500E+00	2.0950E+00	2.692E-02
15	3.7500E+00	1.5065E+00	2.810E-02
16	4.2500E+00	1.3408E+00	3.226E-02
17	4.7500E+00	1.3009E+00	2.885E-02
18	5.2500E+00	1.2328E+00	2.832E-02
19	5.7500E+00	1.1960E+00	3.051E-02
20	6.2500E+00	1.1615E+00	2.558E-02
21	6.7500E+00	1.1462E+00	2.486E-02
22	7.2500E+00	1.1107E+00	1.881E-02
23	7.7500E+00	1.1096E+00	2.688E-02
24	8.2500E+00	1.1238E+00	2.141E-02
25	8.7500E+00	1.1248E+00	3.378E-02
26	9.2500E+00	1.1167E+00	2.694E-02
27	9.7500E+00	1.1115E+00	2.602E-02
28	1.0250E+01	1.0899E+00	2.220E-02
29	1.0750E+01	1.0539E+00	5.670E-02
30	1.1250E+01	1.0536E+00	2.128E-02
31	1.1750E+01	1.0479E+00	5.137E-02
32	1.2250E+01	1.0233E+00	3.388E-02
33	1.2750E+01	1.0175E+00	5.595E-02
34	1.3250E+01	9.9159E-01	4.170E-02
35	1.3750E+01	9.8367E-01	5.539E-02
36	1.4250E+01	9.9912E-01	1.360E-02
37	1.4750E+01	9.7955E-01	3.053E-02
38	1.5500E+01	9.4118E-01	3.689E-02
39	1.6500E+01	9.5640E-01	1.715E-01
40	1.7500E+01	9.4001E-01	4.251E-02
41	1.8500E+01	8.9934E-01	5.475E-02
42	1.9500E+01	8.8711E-01	3.777E-02

22650004

MT = 3, printed from be9mt3.dat

	Neutron energy (in MeV)	cross section (in barn)	std.dev. of sigma(E) (in barn)
1	5.0000E-01	0.0000E+00	0.000E+00
2	7.7500E-01	1.3482E-05	2.083E-05
3	9.2500E-01	2.2563E-04	4.623E-05
4	1.1000E+00	9.0355E-04	1.268E-04
5	1.3000E+00	5.3745E-03	5.922E-04
6	1.5000E+00	1.3834E-02	1.469E-03
7	1.7000E+00	2.4755E-02	1.859E-03
8	1.9000E+00	3.7771E-02	2.269E-03
9	2.1000E+00	5.4100E-02	2.942E-03
10	2.3000E+00	7.5643E-02	3.199E-03
11	2.5000E+00	1.0016E-01	3.951E-03
12	2.7000E+00	1.2929E-01	5.388E-03
13	2.9000E+00	3.7041E-01	1.479E-02
14	3.2500E+00	5.3495E-01	2.320E-02
15	3.7500E+00	5.9170E-01	2.621E-02
16	4.2500E+00	5.8704E-01	3.071E-02
17	4.7500E+00	5.9656E-01	2.845E-02
18	5.2500E+00	6.2986E-01	2.824E-02
19	5.7500E+00	6.5245E-01	3.048E-02
20	6.2500E+00	6.4819E-01	2.549E-02
21	6.7500E+00	6.2824E-01	2.473E-02
22	7.2500E+00	6.3360E-01	1.833E-02
23	7.7500E+00	6.2647E-01	2.701E-02
24	8.2500E+00	5.9693E-01	2.127E-02
25	8.7500E+00	5.8187E-01	3.390E-02
26	9.2500E+00	5.7509E-01	2.734E-02
27	9.7500E+00	5.6873E-01	2.607E-02
28	1.0250E+01	5.6948E-01	2.269E-02
29	1.0750E+01	5.8752E-01	5.671E-02
30	1.1250E+01	5.6435E-01	2.162E-02
31	1.1750E+01	5.4068E-01	5.082E-02
32	1.2250E+01	5.4111E-01	3.406E-02
33	1.2750E+01	5.3599E-01	5.578E-02
34	1.3250E+01	5.3331E-01	4.165E-02
35	1.3750E+01	5.2203E-01	5.537E-02
36	1.4250E+01	4.9178E-01	1.209E-02
37	1.4750E+01	4.8785E-01	2.983E-02
38	1.5500E+01	5.0122E-01	3.668E-02
39	1.6500E+01	4.6910E-01	1.714E-01
40	1.7500E+01	4.5420E-01	4.240E-02
41	1.8500E+01	4.5005E-01	5.439E-02
42	1.9500E+01	4.3399E-01	3.788E-02

22650005

MT = 16, printed from n2nstep2.dat

	Neutron energy (in MeV)	cross section (in barn)	std.dev. of sigma(E) (in barn)
1	1.7488E+00	0.0000E+00	0.000E+00
2	2.0000E+00	2.2750E-03	2.543E-04
3	2.2000E+00	8.4568E-03	7.843E-04
4	2.4000E+00	1.7621E-02	1.139E-03
5	2.6000E+00	2.7543E-02	1.625E-03
6	2.7000E+00	3.6554E-02	2.504E-03
7	2.8000E+00	1.9605E-01	1.018E-02
8	3.0000E+00	3.4159E-01	2.288E-02
9	3.2000E+00	4.2409E-01	2.242E-02
10	3.4000E+00	4.6990E-01	2.777E-02
11	3.6000E+00	4.8614E-01	2.544E-02
12	3.8000E+00	5.0457E-01	2.587E-02
13	4.0000E+00	5.0534E-01	3.047E-02
14	4.5000E+00	5.1479E-01	2.774E-02
15	5.0000E+00	5.4519E-01	2.740E-02
16	5.5000E+00	5.9475E-01	3.050E-02
17	6.0000E+00	6.0981E-01	2.603E-02
18	6.5000E+00	6.0015E-01	2.526E-02
19	7.0000E+00	5.8115E-01	1.867E-02
20	7.5000E+00	6.1635E-01	2.831E-02
21	8.0000E+00	5.7242E-01	2.217E-02
22	8.5000E+00	5.6449E-01	3.463E-02
23	9.0000E+00	5.4841E-01	2.776E-02
24	9.5000E+00	5.5482E-01	2.696E-02
25	1.0000E+01	5.3940E-01	2.277E-02
26	1.0500E+01	5.5725E-01	5.584E-02
27	1.1000E+01	5.6832E-01	2.700E-02
28	1.1500E+01	5.2120E-01	5.029E-02
29	1.2000E+01	5.3026E-01	3.488E-02
30	1.2500E+01	5.1370E-01	5.592E-02
31	1.3000E+01	5.1287E-01	4.244E-02
32	1.3500E+01	5.0476E-01	5.621E-02
33	1.4000E+01	4.8208E-01	9.453E-03
34	1.4500E+01	4.6168E-01	4.857E-02
35	1.5000E+01	4.7590E-01	3.849E-02
36	1.6000E+01	4.5514E-01	5.381E-02
37	1.7000E+01	4.0810E-01	4.215E-02
38	1.8000E+01	4.1266E-01	5.583E-02
39	1.9000E+01	3.9060E-01	3.841E-02
40	2.0000E+01	3.7996E-01	3.739E-02

22650006

MT = 104, printed from ndstep2.dat

	Neutron energy (in MeV)	cross section (in barn)	std.dev. of sigma(E) (in barn)
1	1.6301E+01	0.0000E+00	0.000E+00
2	1.6560E+01	9.9979E-05	1.071E-05
3	1.6700E+01	5.5988E-04	5.995E-05
4	1.6840E+01	1.3797E-03	1.477E-04
5	1.6980E+01	2.7293E-03	2.922E-04
6	1.7120E+01	4.0594E-03	3.207E-04
7	1.7260E+01	5.8792E-03	4.644E-04
8	1.7400E+01	6.4991E-03	5.134E-04
9	1.7540E+01	6.9990E-03	5.529E-04
10	1.7680E+01	7.7989E-03	6.161E-04
11	1.7820E+01	8.1988E-03	6.477E-04
12	1.7960E+01	8.8987E-03	7.030E-04
13	1.8100E+01	9.1987E-03	7.345E-04
14	1.8240E+01	9.7986E-03	7.824E-04
15	1.8380E+01	9.8986E-03	7.904E-04
16	1.8520E+01	9.9986E-03	7.984E-04
17	1.8660E+01	1.0499E-02	8.383E-04
18	1.8800E+01	1.0898E-02	8.702E-04
19	1.9500E+01	1.1895E-02	1.637E-03
20	2.0000E+01	1.2395E-02	1.705E-03

MT = 103, printed from npstep2.dat

	Neutron energy (in MeV)	cross section (in barn)	std.dev. of sigma(E) (in barn)
1	1.4270E+01	0.0000E+00	0.000E+00
2	1.4750E+01	1.1209E-04	1.023E-05
3	1.5500E+01	5.9442E-04	5.722E-05
4	1.6500E+01	8.8328E-04	4.083E-04
5	1.7500E+01	1.0329E-03	5.015E-04
6	1.8500E+01	1.0255E-03	3.667E-04
7	1.9500E+01	1.2026E-03	6.372E-04
8	2.0000E+01	1.2026E-03	6.372E-04

22650007

MT = 105, printed from ntstep2.dat

	Neutron energy (in MeV)	cross section (in barn)	std.dev. of sigma(E) (in barn)
1	1.1608E+01	0.0000E+00	0.000E+00
2	1.2250E+01	4.3418E-03	5.087E-04
3	1.2750E+01	1.0230E-02	7.818E-04
4	1.3250E+01	1.5762E-02	8.581E-04
5	1.3750E+01	1.7291E-02	8.059E-04
6	1.4250E+01	1.8901E-02	7.946E-04
7	1.4750E+01	2.3971E-02	1.066E-03
8	1.5500E+01	2.9564E-02	1.594E-03
9	1.6500E+01	2.8722E-02	1.614E-03
10	1.7500E+01	2.8931E-02	1.583E-03
11	1.8500E+01	3.0915E-02	1.738E-03
12	1.9500E+01	2.9928E-02	1.734E-03
13	2.0000E+01	2.9928E-02	1.734E-03

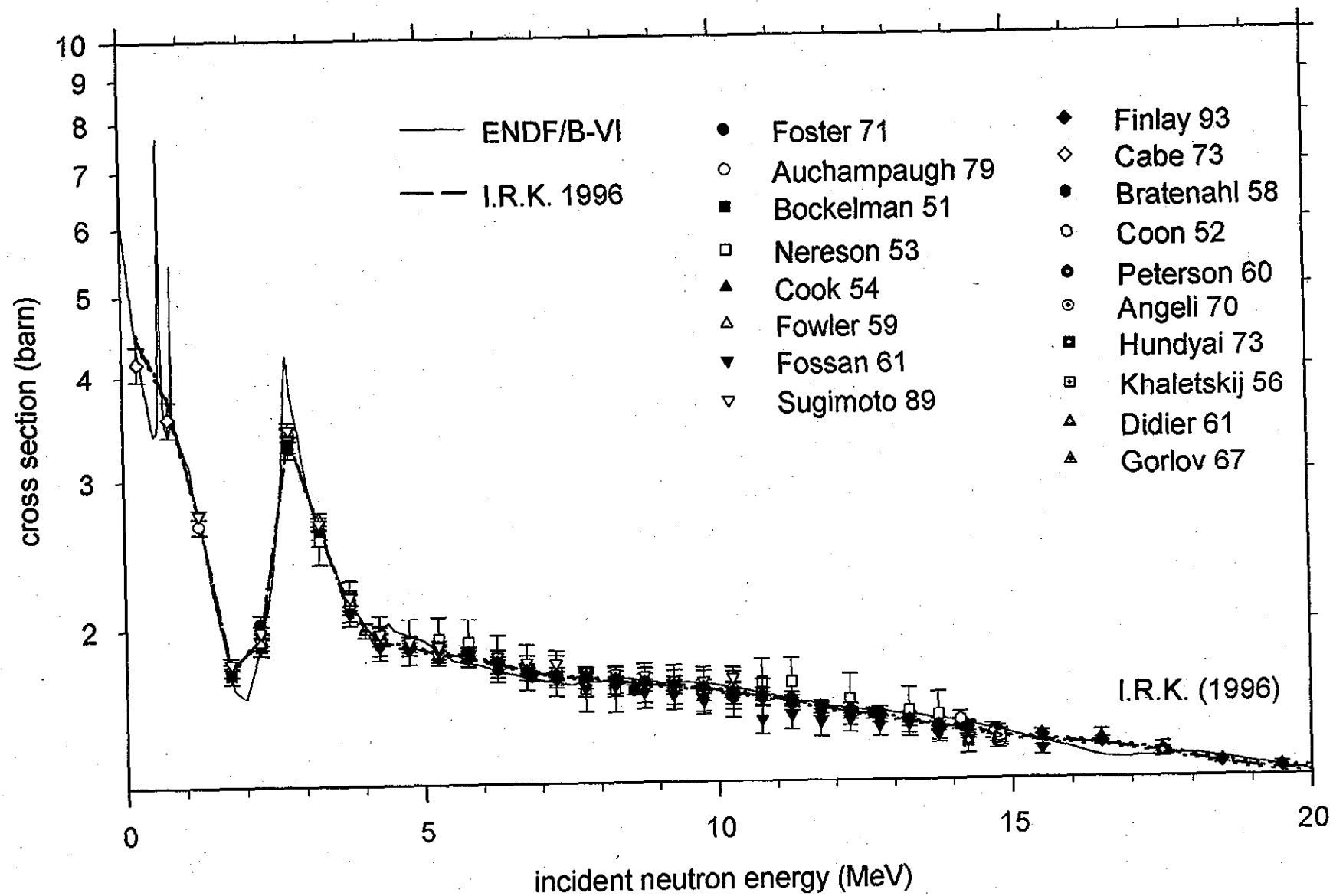
22650008

MT = 107, printed from nstep2.dat

	Neutron energy (in MeV)	cross section (in barn)	std.dev. of sigma(E) (in barn)
1	6.6715E-01	0.0000E+00	0.0000E+00
2	8.3360E-01	2.0811E-05	3.230E-05
3	9.0000E-01	1.4519E-04	2.986E-05
4	1.0000E+00	4.6694E-04	7.955E-05
5	1.1000E+00	9.0355E-04	1.271E-04
6	1.2000E+00	3.2364E-03	4.240E-04
7	1.3000E+00	5.3745E-03	5.937E-04
8	1.4000E+00	8.8068E-03	9.379E-04
9	1.6000E+00	1.8861E-02	1.420E-03
10	1.8000E+00	3.0647E-02	1.909E-03
11	2.0000E+00	4.2153E-02	2.496E-03
12	2.2000E+00	5.5309E-02	2.628E-03
13	2.4000E+00	6.9883E-02	3.330E-03
14	2.6000E+00	8.5240E-02	4.422E-03
15	2.8000E+00	1.0019E-01	5.270E-03
16	3.0000E+00	1.0260E-01	4.665E-03
17	3.5000E+00	9.5602E-02	5.058E-03
18	4.0000E+00	8.7310E-02	4.606E-03
19	4.5000E+00	6.5741E-02	3.395E-03
20	5.0000E+00	6.5843E-02	7.077E-03
21	5.5000E+00	5.2089E-02	6.304E-03
22	6.0000E+00	4.6376E-02	6.510E-03
23	6.5000E+00	3.8332E-02	6.219E-03
24	7.0000E+00	3.4502E-02	5.811E-03
25	7.5000E+00	3.2728E-02	5.711E-03
26	8.0000E+00	2.8702E-02	5.838E-03
27	8.5000E+00	2.5438E-02	6.015E-03
28	9.0000E+00	2.2582E-02	5.929E-03
29	9.5000E+00	2.0972E-02	5.568E-03
30	1.0000E+01	1.9393E-02	5.208E-03
31	1.0500E+01	1.7872E-02	4.798E-03
32	1.1000E+01	1.6330E-02	4.347E-03
33	1.1500E+01	1.4825E-02	3.923E-03
34	1.2000E+01	1.4655E-02	3.111E-03
35	1.2500E+01	1.3047E-02	2.992E-03
36	1.3000E+01	1.1597E-02	2.953E-03
37	1.3500E+01	1.0868E-02	2.752E-03
38	1.4000E+01	9.7417E-03	6.927E-04
39	1.4500E+01	9.4777E-03	2.293E-03
40	1.5000E+01	8.9787E-03	2.538E-03
41	1.6000E+01	7.9964E-03	2.434E-03
42	1.7000E+01	7.1870E-03	2.258E-03
43	1.8000E+01	6.4277E-03	2.026E-03
44	1.9000E+01	5.5741E-03	1.757E-03
45	2.0000E+01	5.1450E-03	1.624E-03

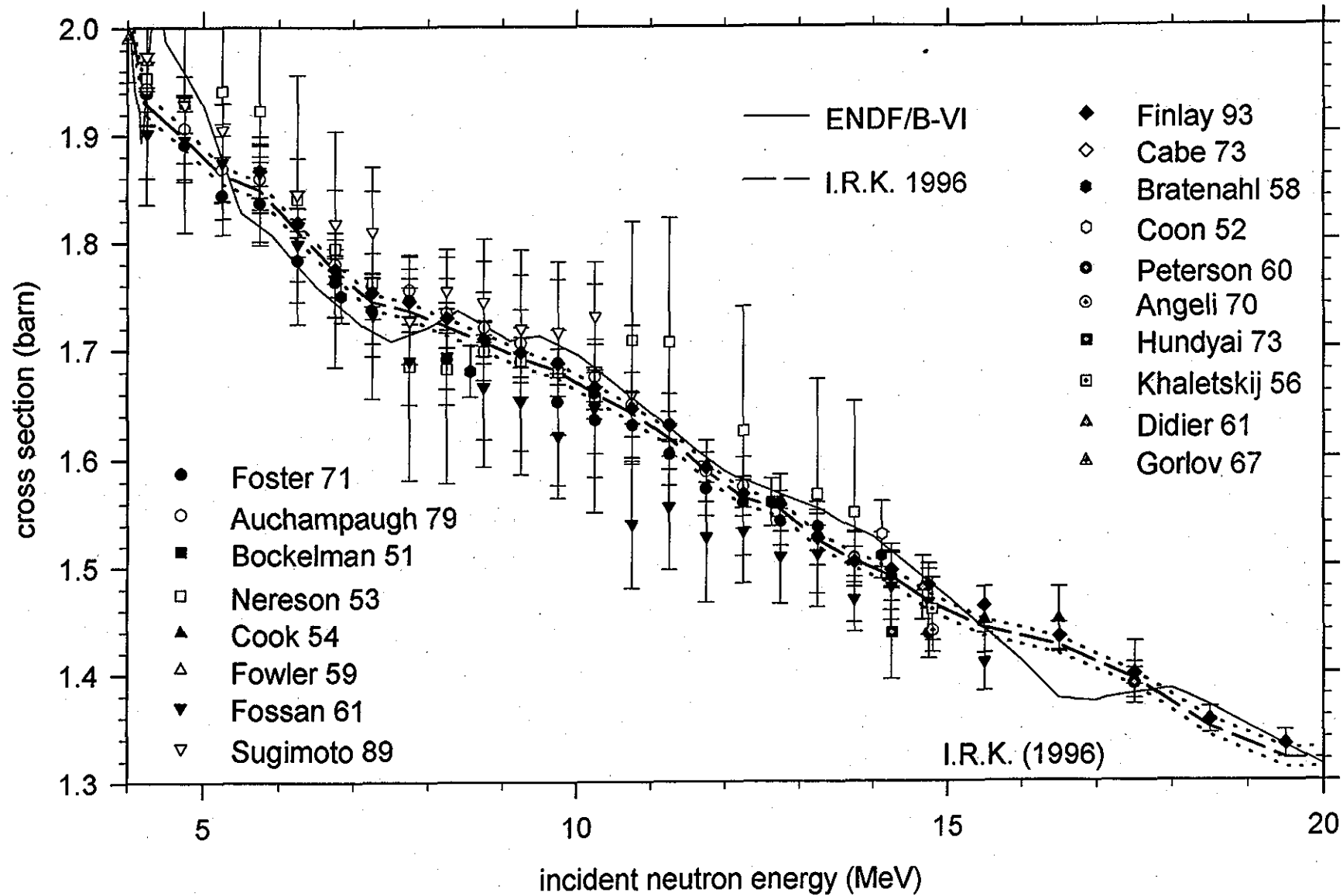
22650009

⁹Be (n,total) final evaluated cross section



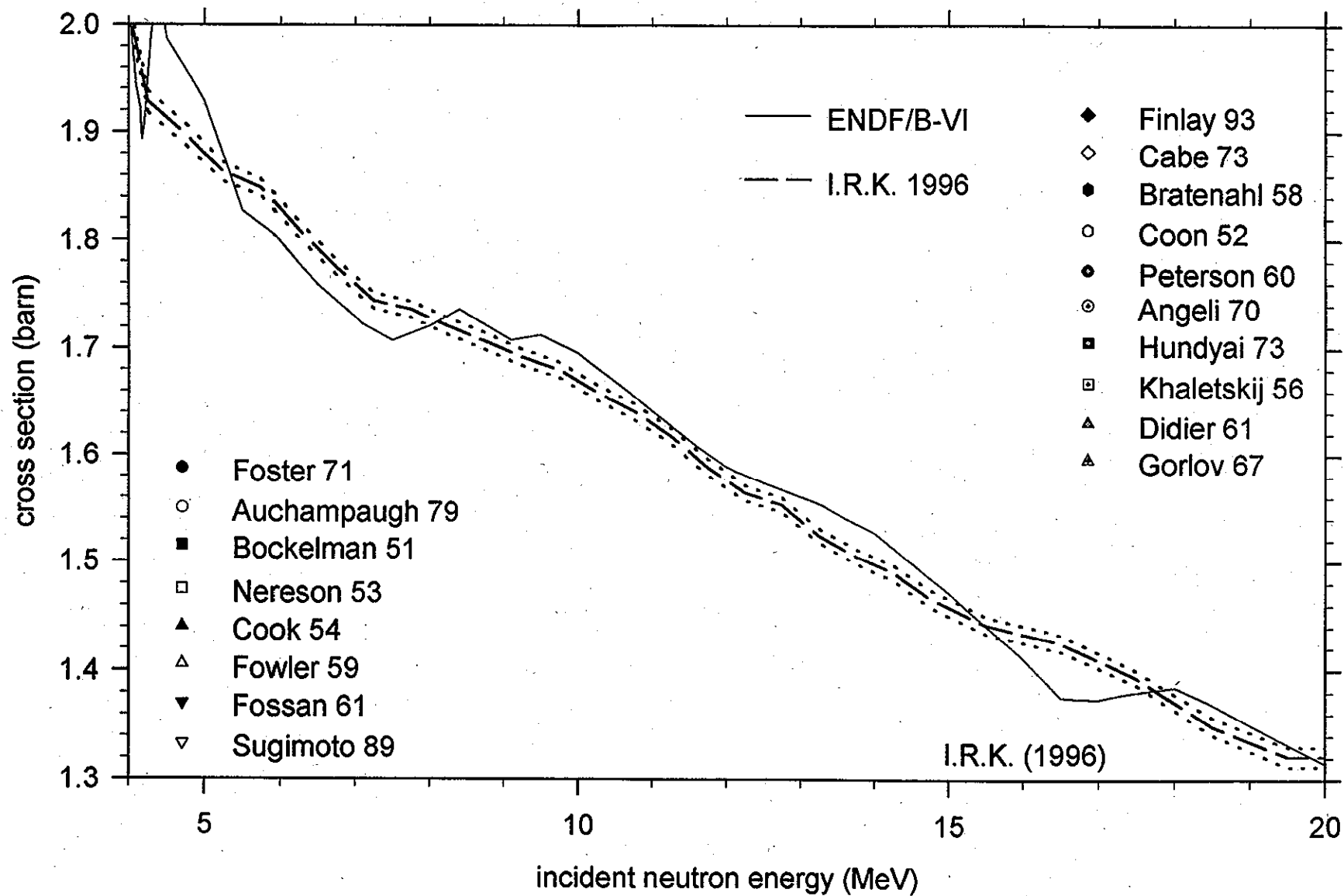
22650010

⁹Be (n,total) final evaluated cross section



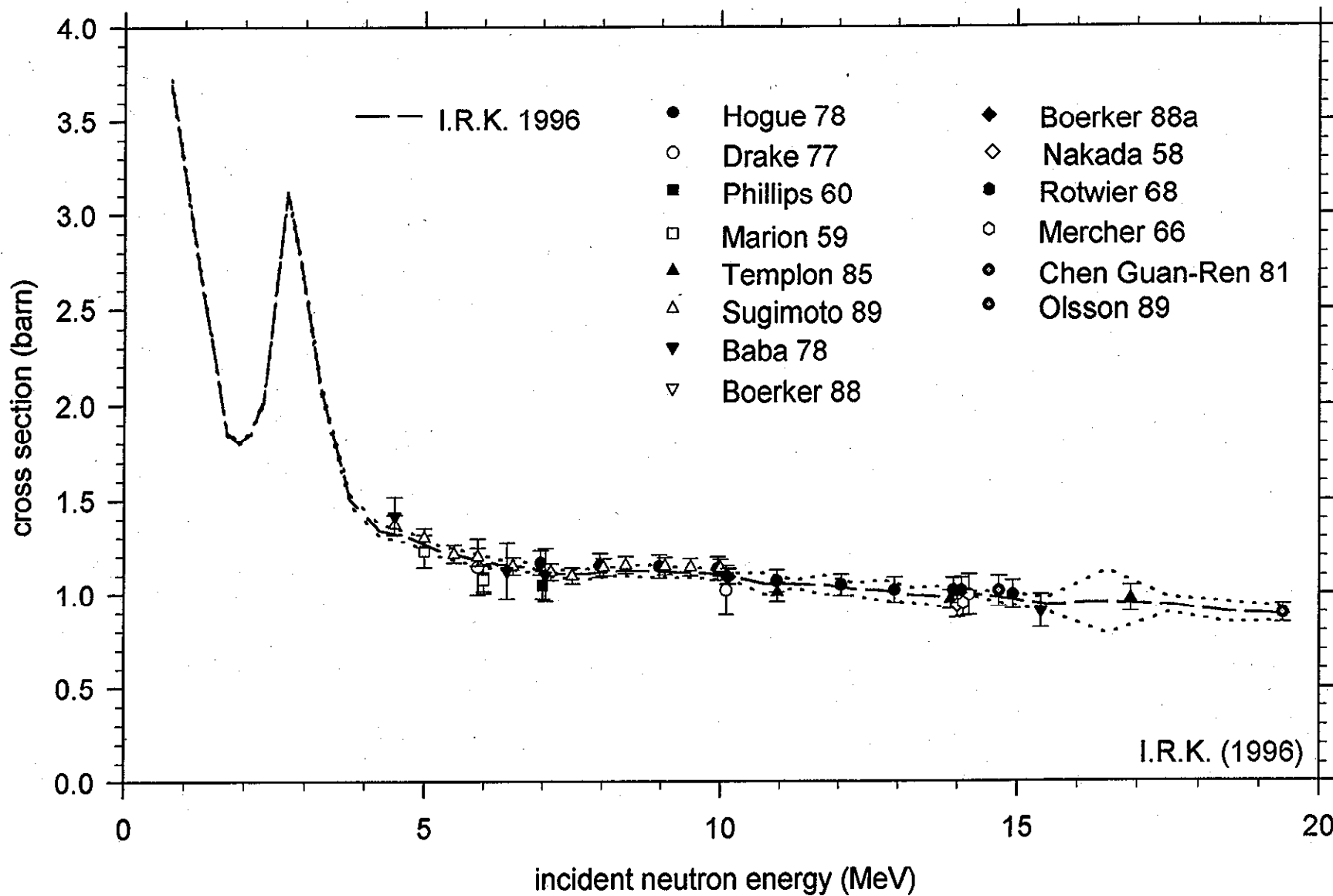
22650011

^9Be (n,total) final evaluated cross section



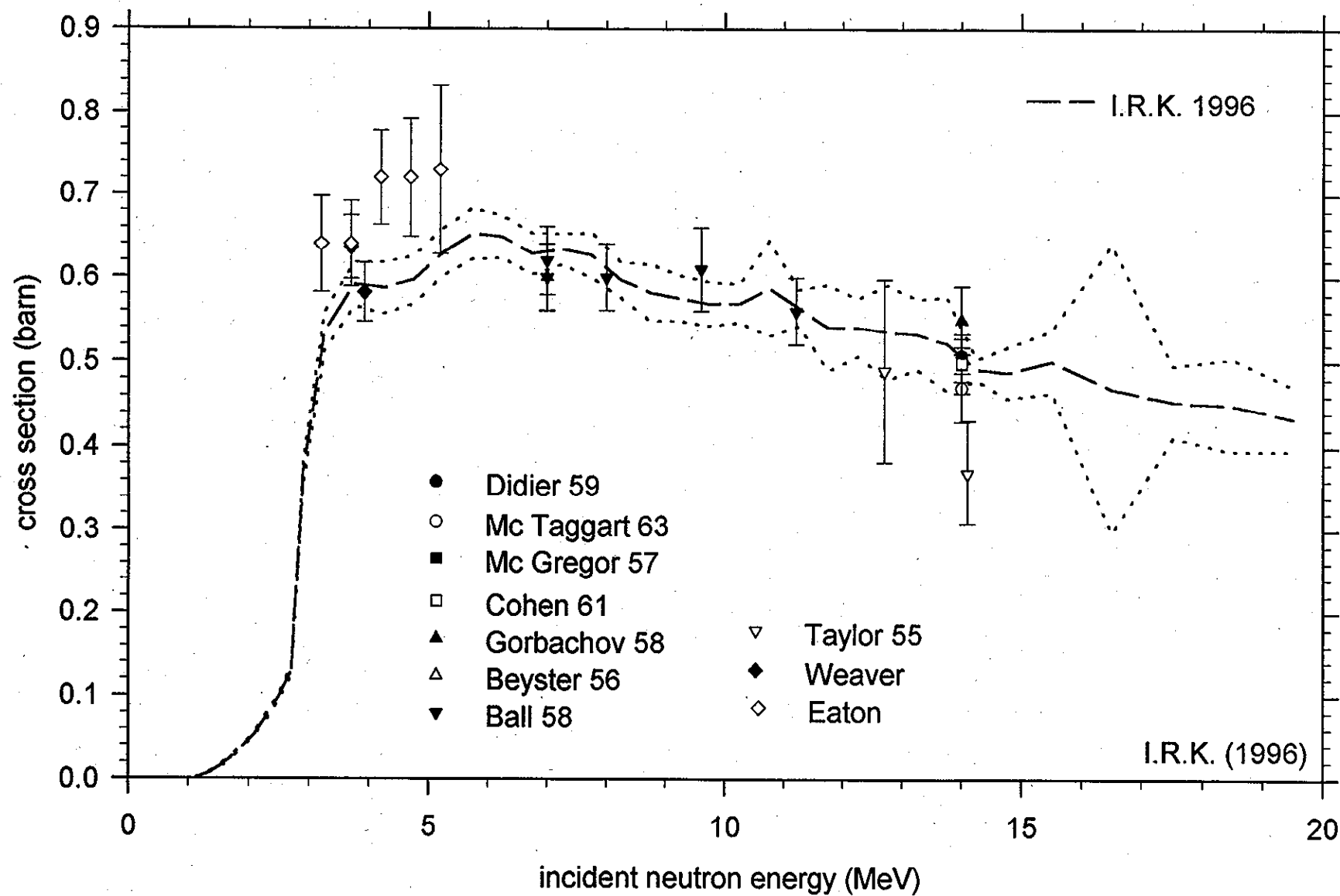
22650012

^9Be (n,elastic) final evaluated cross section



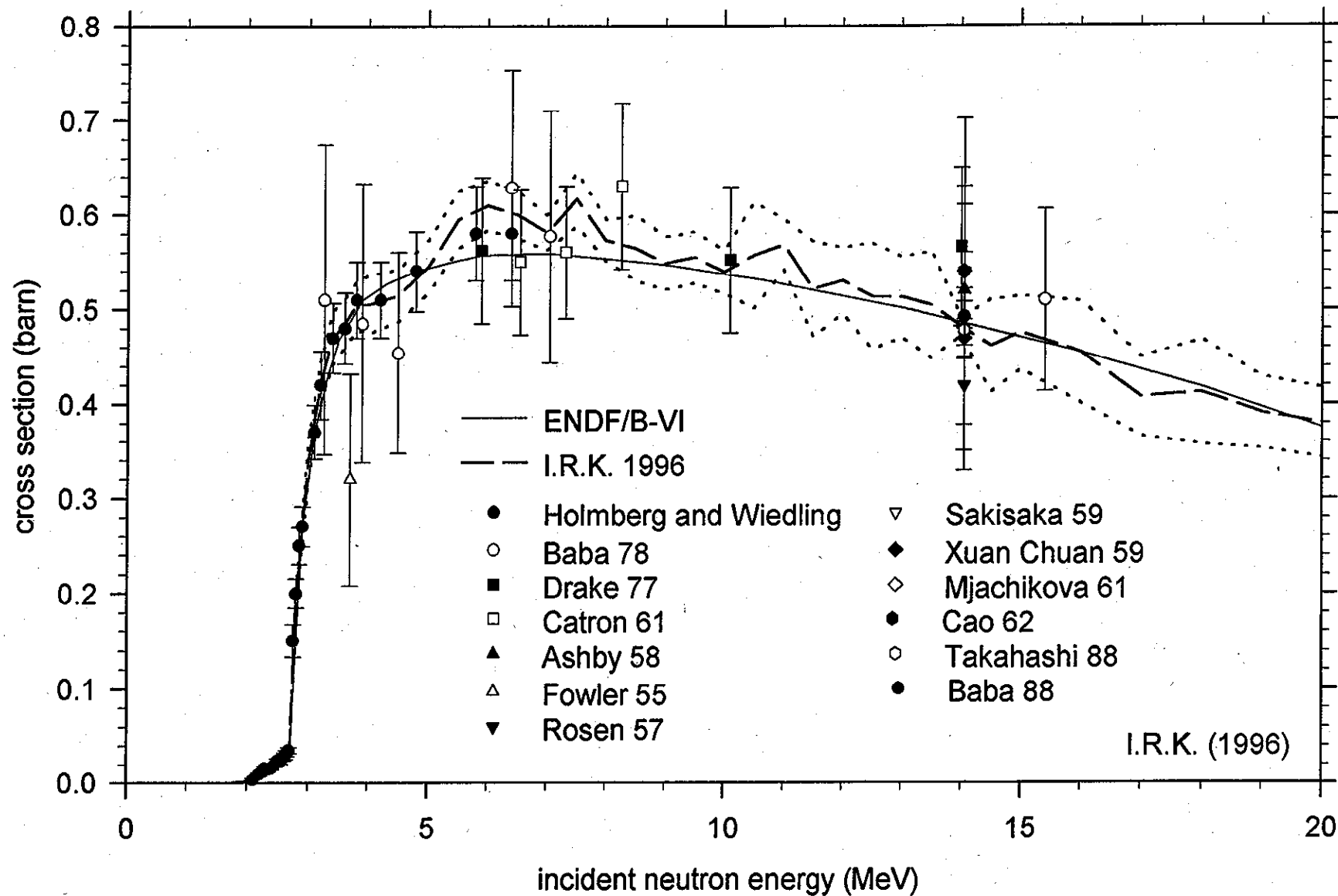
22650013

⁹Be (n,nonelastic) final evaluated cross section



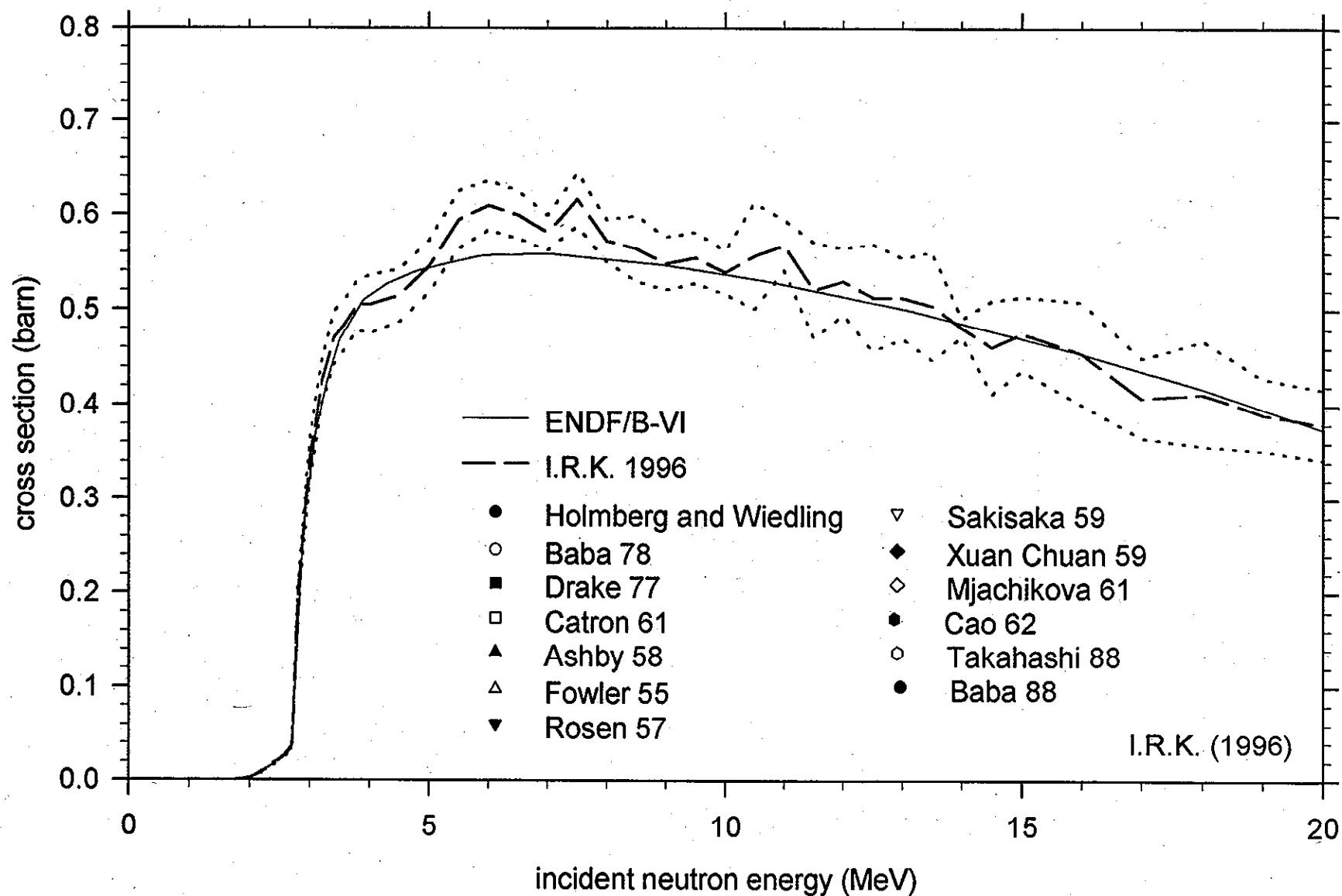
22650014

⁹Be (n,2n) final evaluated cross section



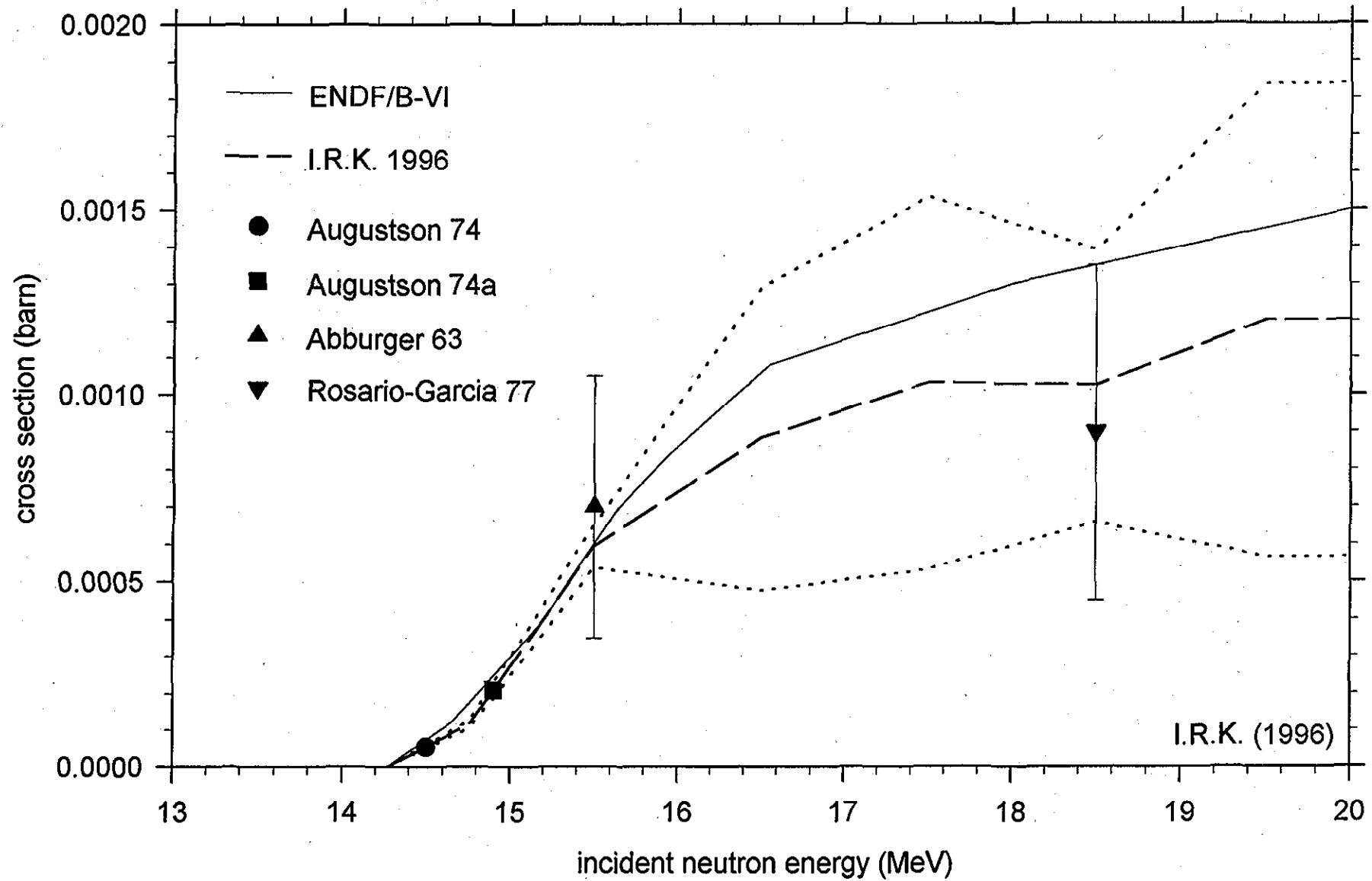
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⁹Be (n,2n) final evaluated cross section



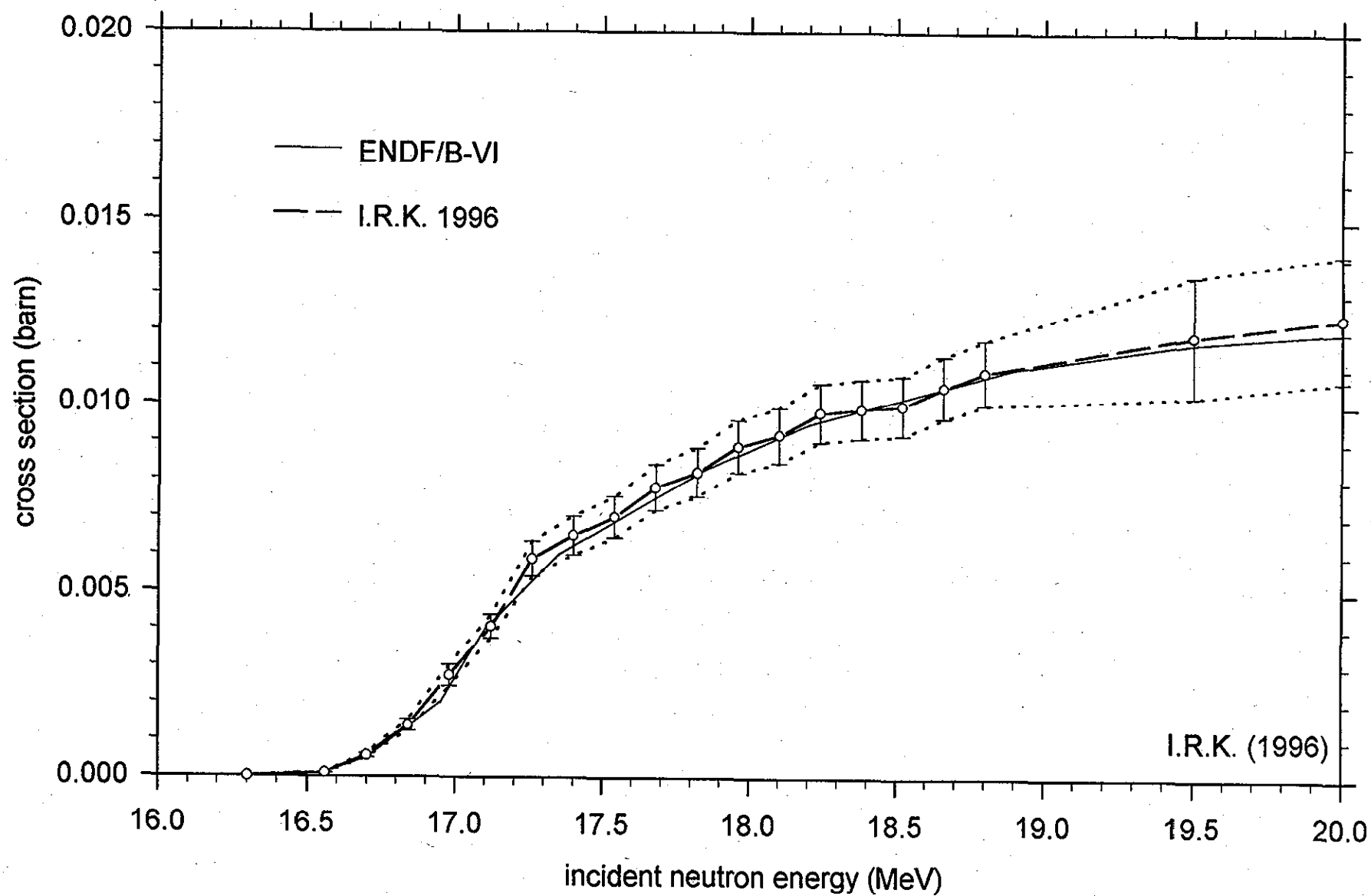
22650016

^9Be (n,p) final evaluated cross section



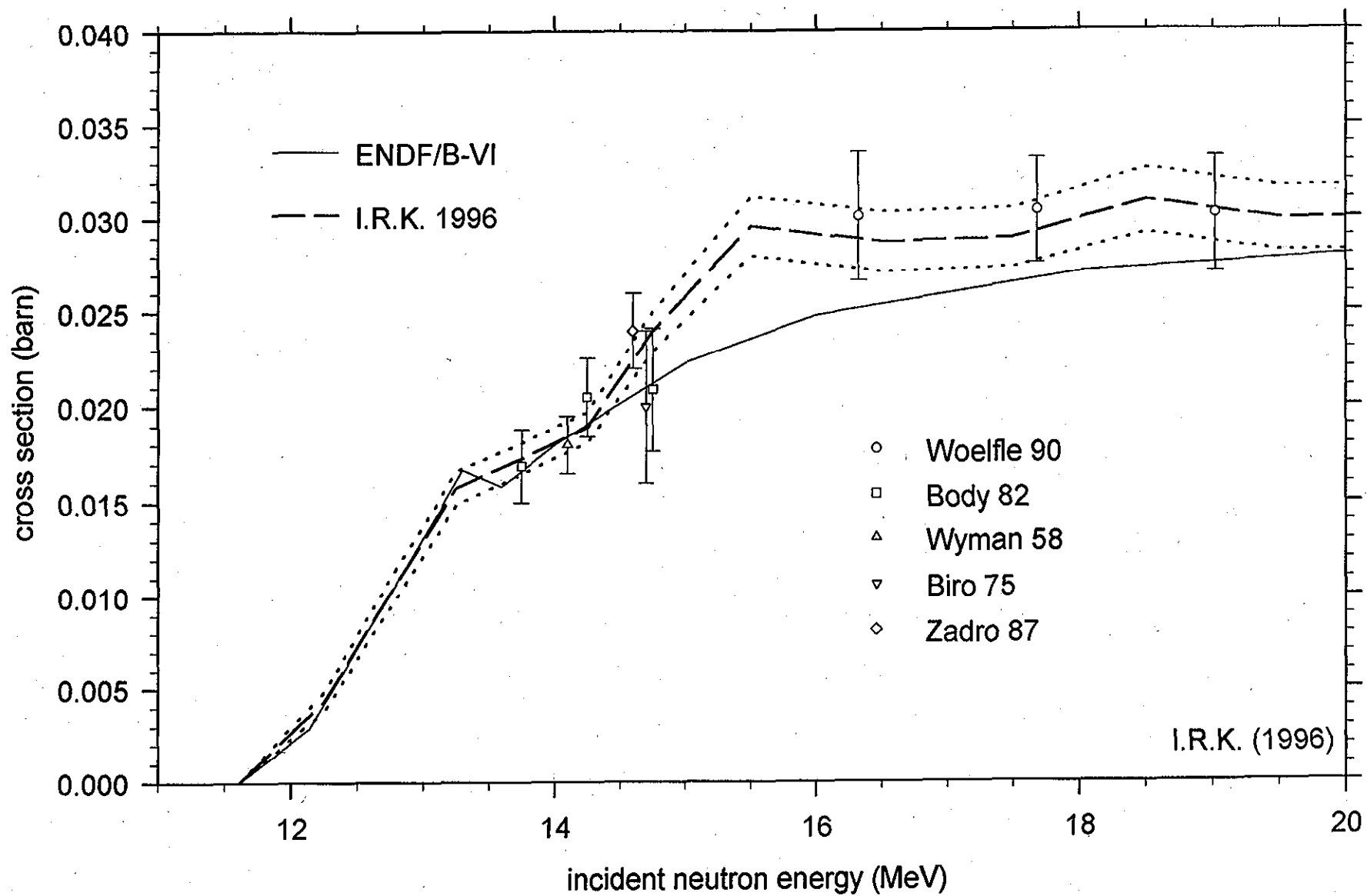
22650017

^9Be (n,d) final evaluated cross section



22650018

^9Be (n,t) final evaluated cross section

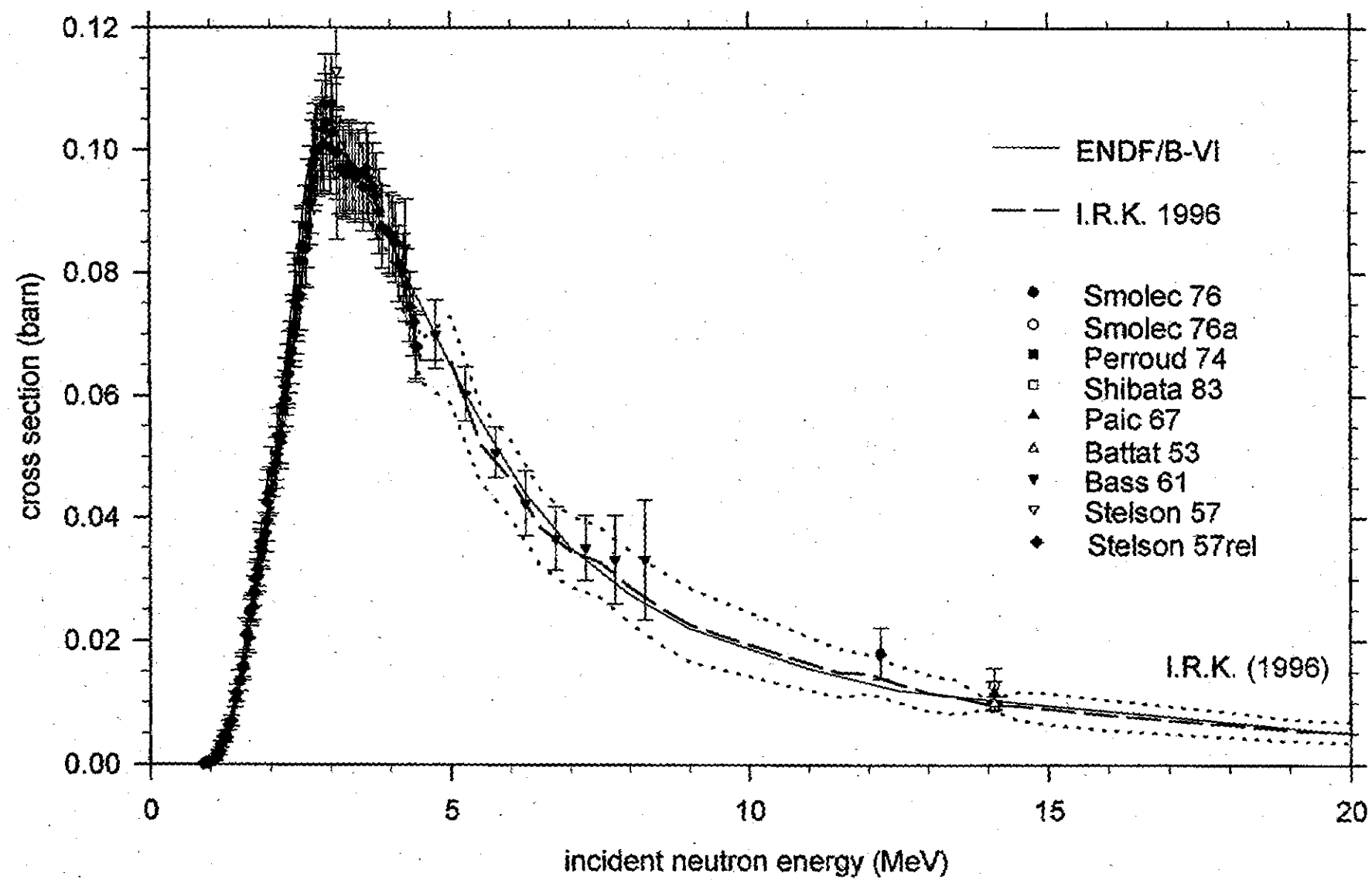


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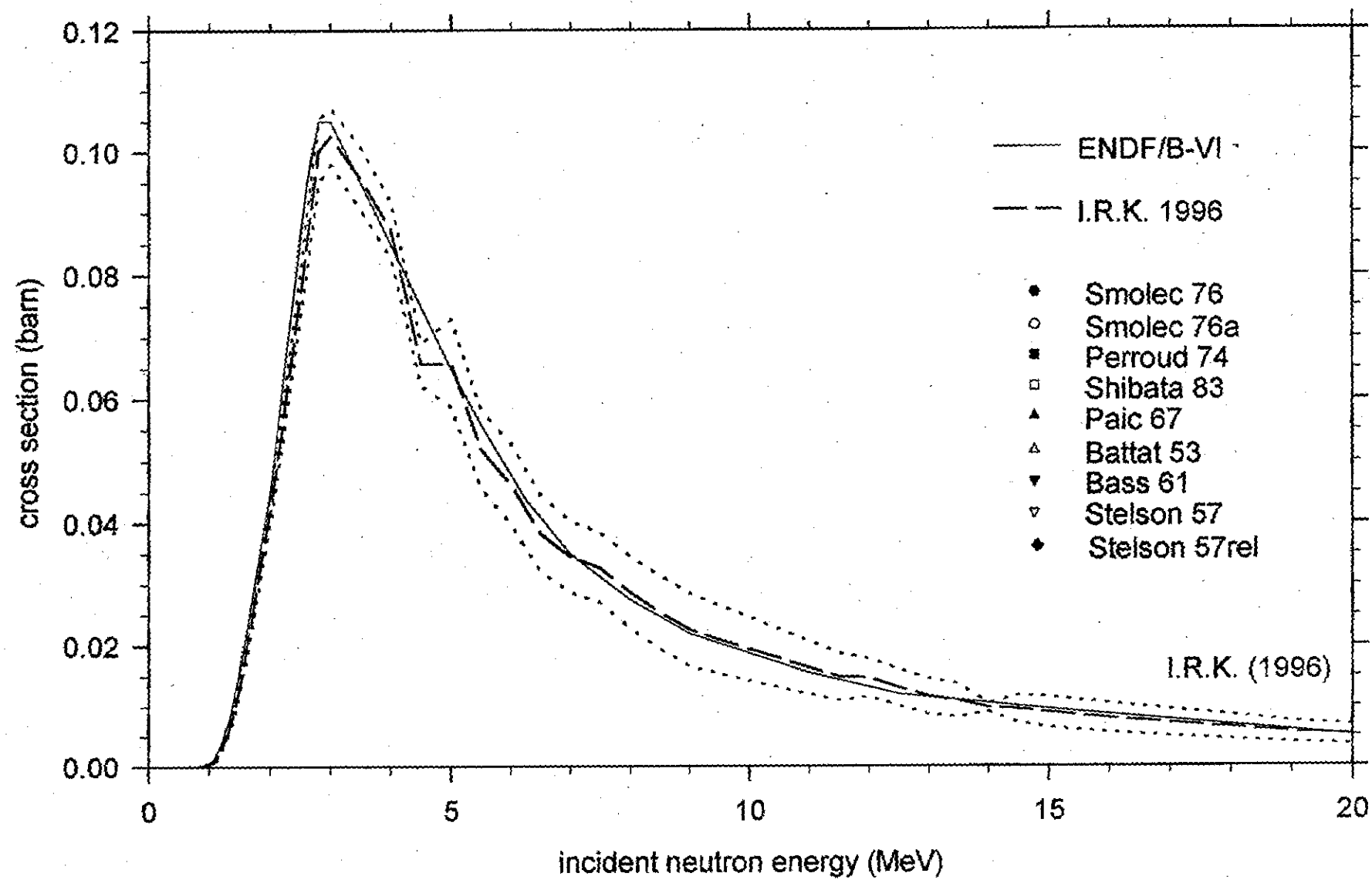
22650019

⁹Be (n,α) final evaluated cross section



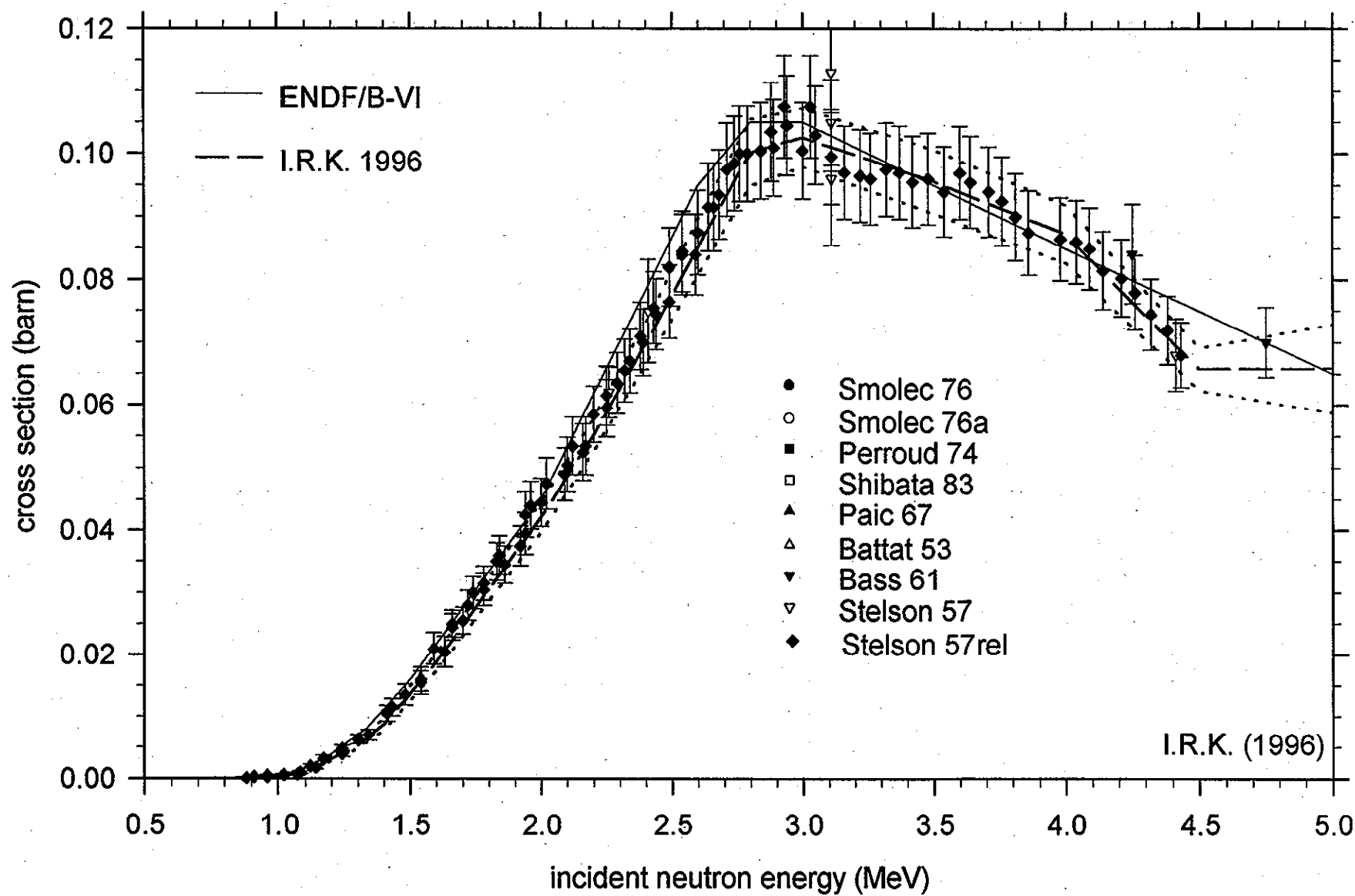
22650020

^9Be (n, α) final evaluated cross section



22650021

^9Be (n, α) final evaluated cross section



I.R.K. (1996)

9-Be (n,2n) MF6, incident energy 2 MeV

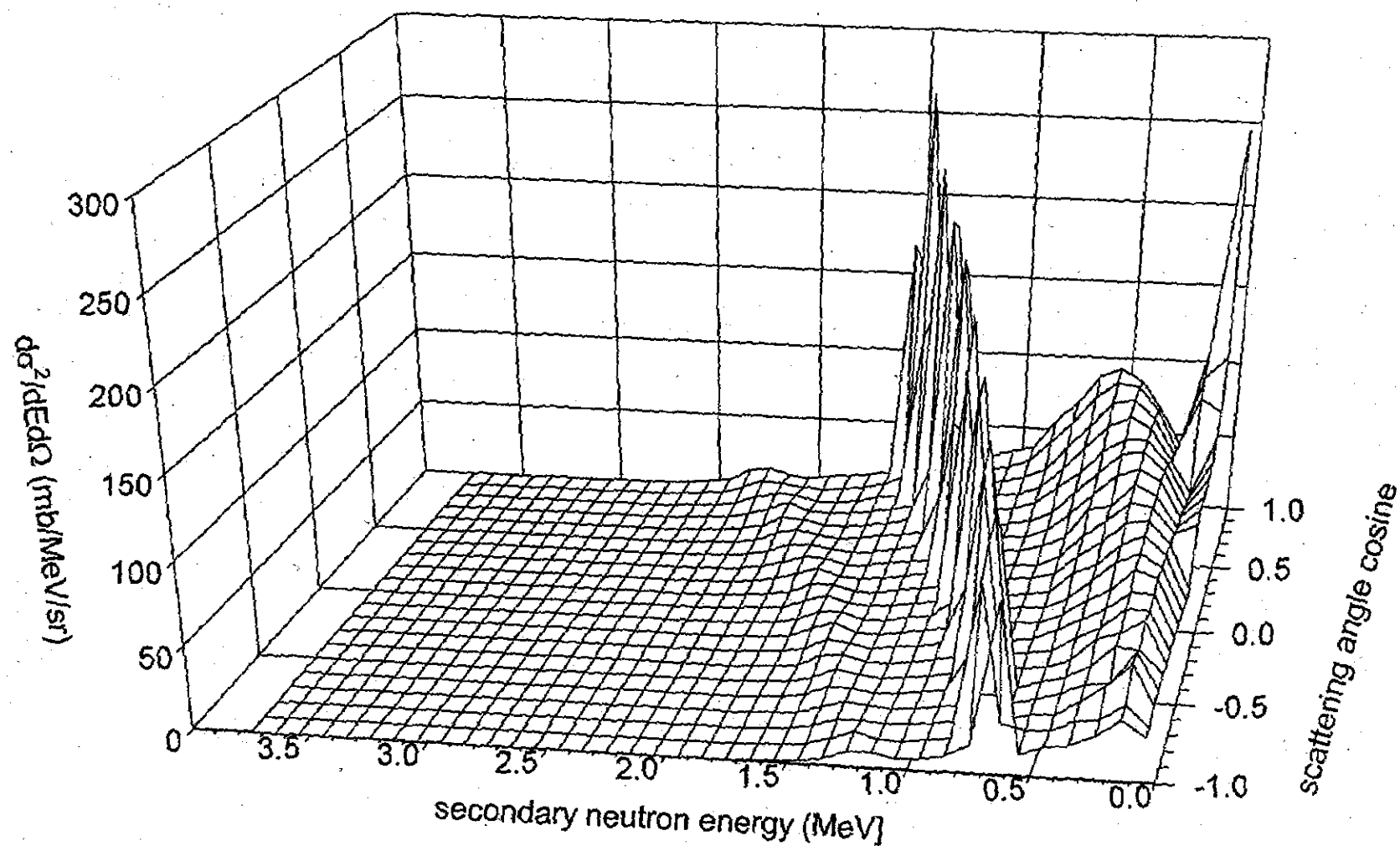
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17	1	0	0	0	0 425 6 16
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0.0000E+00	6.8780E-06	-1.0000E+00	2.4938E-02	-9.0000E-01	6.0279E-03 425 6 16
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-5.0000E-01	3.7304E-02	-4.0000E-01	6.3267E-02	-3.0000E-01	6.5151E-02 425 6 16
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4.0000E-01	1.0128E-01	5.0000E-01	1.1134E-01	6.0000E-01	1.2271E-01 425 6 16
7.0000E-01	1.1578E+00	8.0000E-01	1.3249E+00	9.0000E-01	1.4896E+00 425 6 16
1.0000E+00	4.5261E-02	1.0000E+05	1.8185E-06	-1.0000E+00	7.8436E-02 425 6 16
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-6.0000E-01	5.9179E-02	-5.0000E-01	6.5912E-02	-4.0000E-01	7.2714E-02 425 6 16
-3.0000E-01	8.0587E-02	-2.0000E-01	8.9323E-02	-1.0000E-01	1.2282E-01 425 6 16
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9.0000E-01	7.2037E-01	1.0000E+00	7.8959E-01	2.0000E+05	7.4402E-07 425 6 16
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8.0000E-01	0.0000E+00	9.0000E-01	0.0000E+00	1.0000E+00	0.0000E+00 425 6 16

22650023

22650023

$^9\text{Be}(n,2n)$ neutron energy angular distributions

incident neutron energy 4.0 MeV

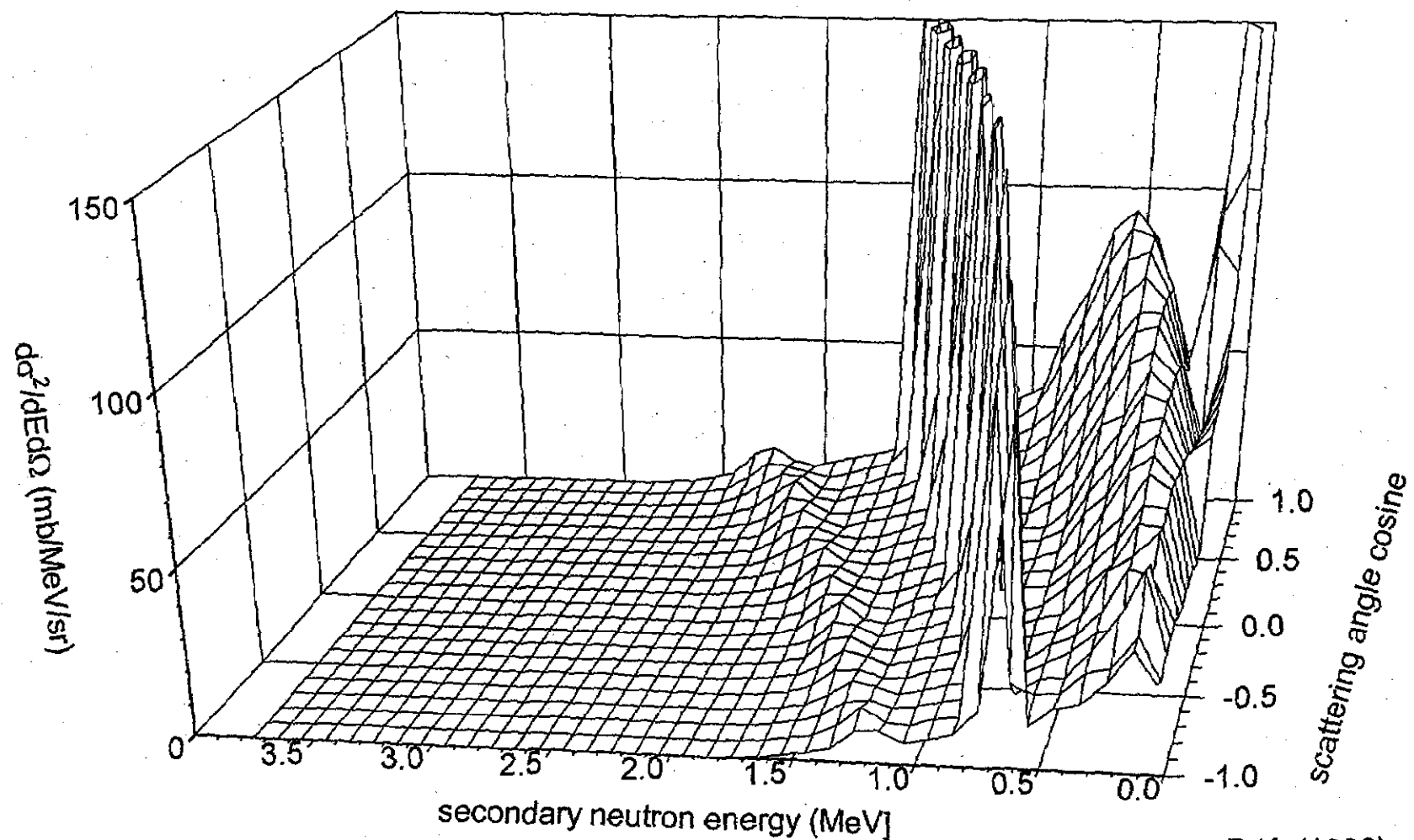


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22650024

$^9\text{Be}(n,2n)$ neutron energy angular distributions

incident neutron energy 4.0 MeV

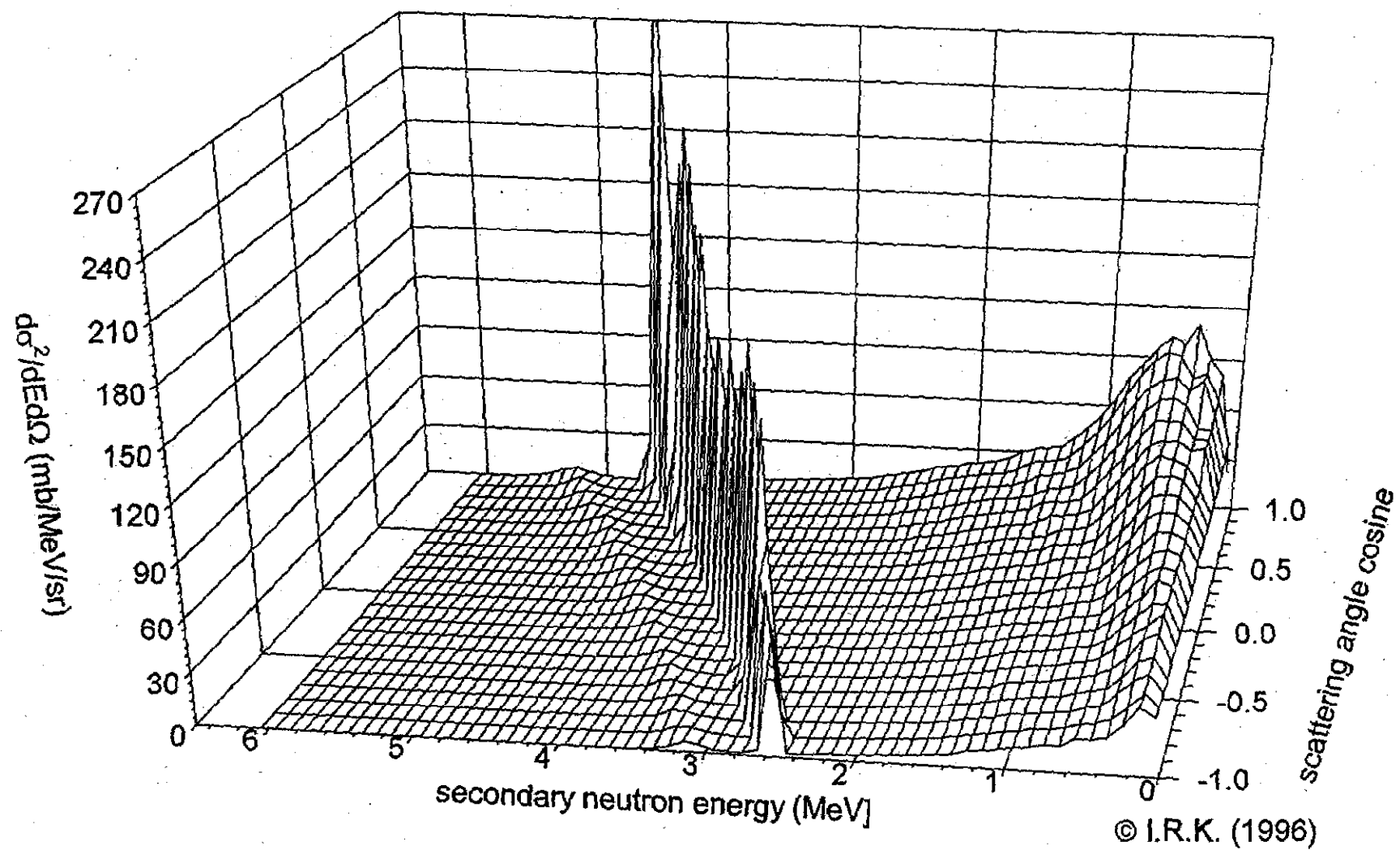


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22650025

$^9\text{Be}(n,2n)$ neutron energy angular distributions

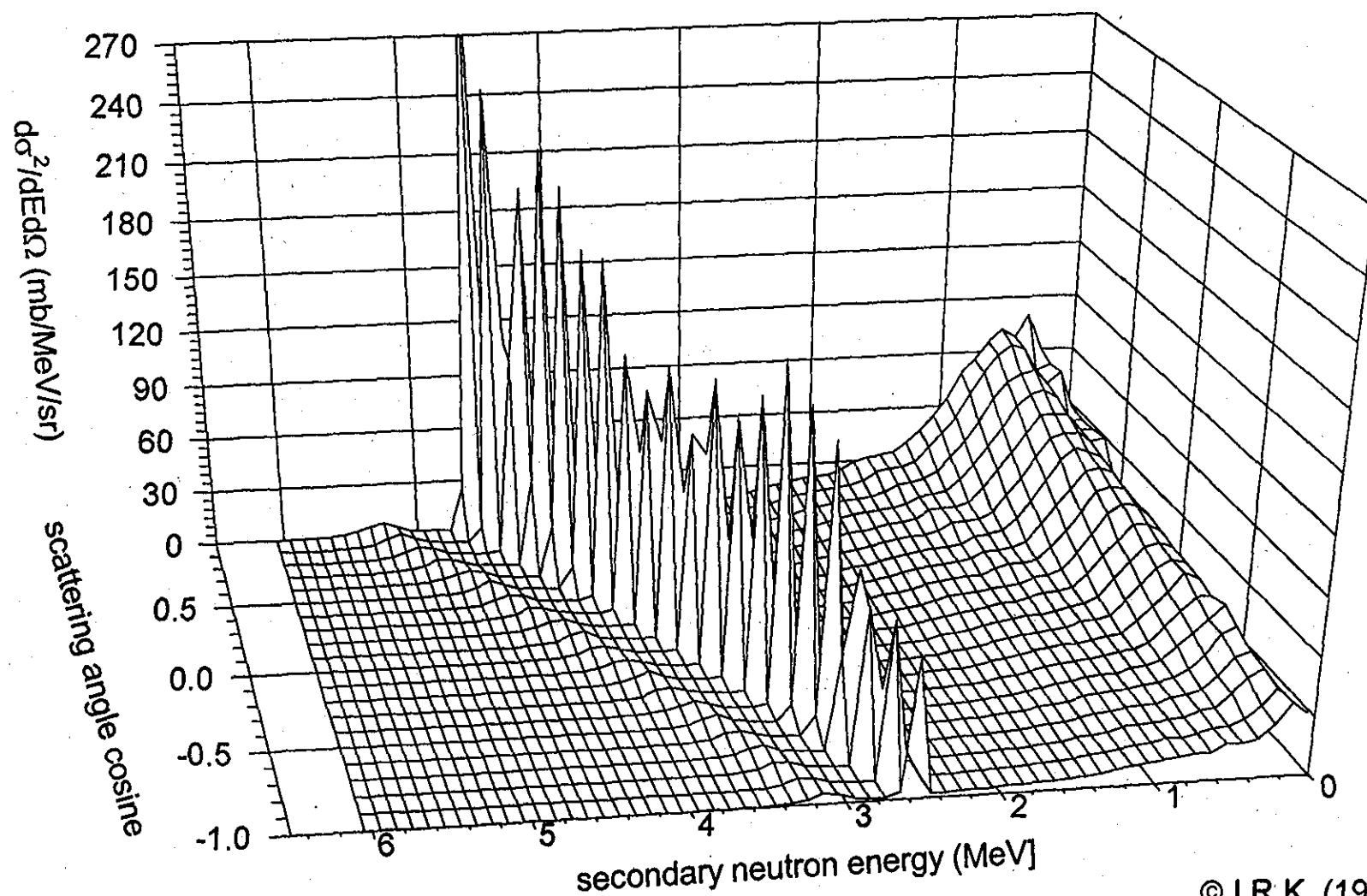
incident neutron energy 7.0 MeV



22650026

$^9\text{Be}(n,2n)$ neutron energy angular distributions

incident neutron energy 7.0 MeV

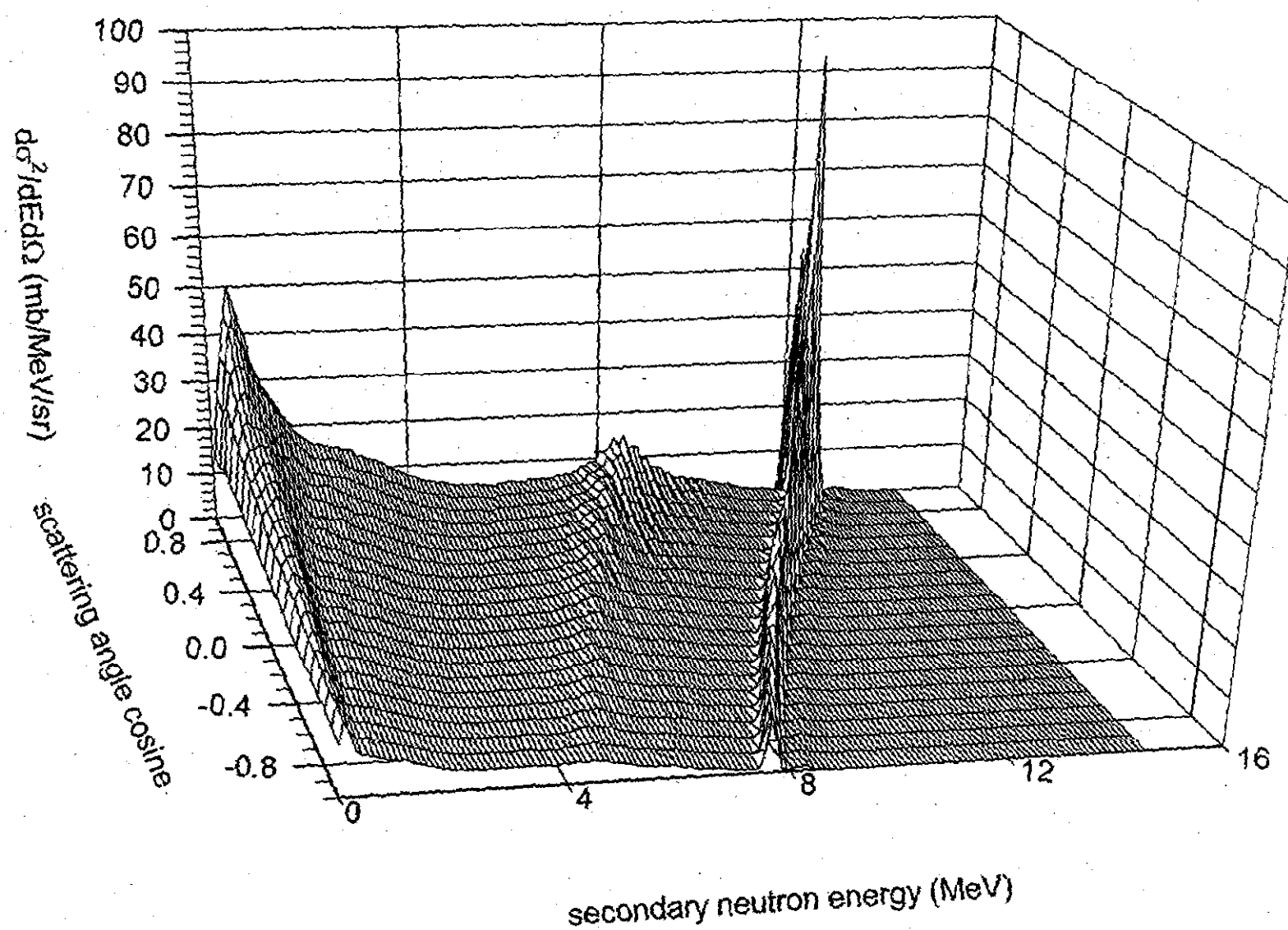


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22650027

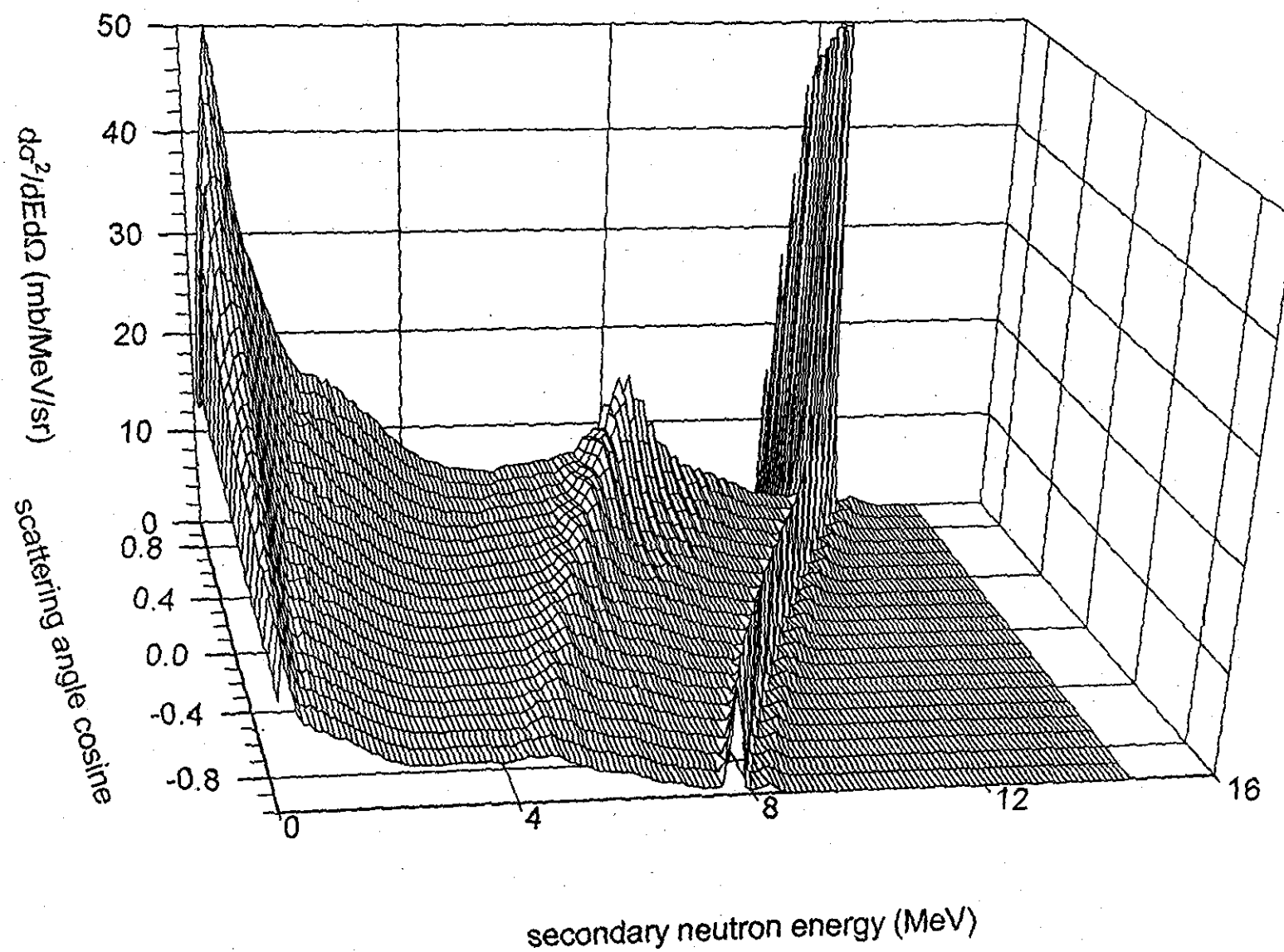
$^9\text{Be}(n,2n)$ neutron energy angular distributions

incident neutron energy 15.0 MeV



22650028

$^9\text{Be}(n,2n)$ neutron energy angular distributions
incident neutron energy 15.0 MeV



22650029